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August 2, 1985

TMI Program Office Attn: Dr. B. J. Snyder Program Director US Nuclear Regulatory Commission Washington, DC 20555

Dear Dr. Snyder:

Three Mile Island Nuclear Station, Unit 2 (TMI-2) Operating License No. DPR-73 Docket No. 50-320 System Description for the Solid Waste Staging Facility

Attached for your information is the annual update to the System Description for the Solid Waste Staging Facility (SWSF) as requested by your letter dated February 4, 1982.

This update reflects minor changes in the operation of the SWSF.

Sincerely. 1 57.

F. R. Standerfer Vice President/Director, TMI-2

FRS/RDW/eml

Attachment

cc: Deputy Program Director - TMI Program Office, Dr. W. D. Travers

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DIVISION

SYSTEM DESCRIPTION

FOR

Solid Waste Staging Facility

DATE 7/18/85 COG ENG RTR 2 COG ENG MGR. _ adam milh __ DATE _7/18/05

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Title Solid	d Waste Staging Facility		Page 2	of 22
Rev.	SUMMARY OF CHANGE	Aş	oproval	Dat
0 In	itial issue per letter 4400-82-L-0059			4/82
1 Re	issued per 4410-83-L-078			3/83
2 Ani	uual Update			7/85

THREE MILE ISLAND NUCLEAR STATION UNIT 2 RECOVERY PROGRAM SOLID WASTE STAGING FACILITY SYSTEM DESCRIPTION

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SYSTEM DESCRIPTION

OF

SOLID WASTE STAGING FACILITY (SWSF)

1.0 INTRODUCTION

The Solid Waste Staging Facility (SWSF) performs no active function. The facility is a passive system for temporary staging of nuclear radioactive waste prior to preparation for shipment and disposal to approved offsite burial grounds. The SWSF has been designed and will be operated in such a manner as to provide assurance that:

- A. The health and safety of the public will be protected.
- B. Occupational exposures will be as low as reasonably achievable (ALARA)
- C. There will be no significant adverse impact on the environment.

2.0 DESIGN DESCRIPTION

2.1 Facility Function

- 2.1.1 The SWSF is used for the collection and temporary staging of the radioactive waste (i.e., Solidified/Dewatered Resins, Filters, Sludges, etc.) generated during the Recovery cleanup operations at TML Unit 2, and operations in Unit 1.
- 2.1.2 The SWSF is located as shown on Figure 1, South and East of Unit 2 Natural Draft Cooling Towers within the area protected by the Flood Control Dikes.

Space was allocated to accomodate six (6) modular structures; two (2) modules "A" and "B" are completed, the remaining space is available for additional modules as, or if, required.

- 2.1.3 Figure 2 shows the typical layout of the SWSF system.
- 2.1.3.1 Each Module is designed with Sixty (60) Cells forming the compartments for storing the radioactive waste generated during processing.

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- 2.1.3.2 Each Cell is designed to stage the radioactive waste contained in either One (1) 6 ft. diameter by 6 ft. high liner, or Two (2) 4 ft. diameter by 4 ft. high liners, or Eighteen (18) 55 gallon, type DOT.17H Drums.
- 2.1.3.3 Each Module is designed to accomodate any combination of the radioactive waste containers as follows:

Container	Total Quantity	Wt of One Filled Container 1b.
5 ft. dia. x 6 ft. H Liners	60 (1 per cell)	7,000 - 22,000 max.
55 gallon drums	1080 (18 per cell)	840
ft. dia. x 4 ft. H Liners	120 (2 per cell)	4,500 - 5,000 max.

2.1.3.4 A floor drainage system is incorporated into the module design which discharges into a common sump located between Modules "A" and "B". The total capacity of the sump is approximately 2750 gallons.

2.2 References

- 2.2.1 U.S. NRC Regulatory Guide 1.143, July 1978, Design Guidance for Radioactive Waste Management Systems, Structures, and Components Installed in Light-Water-Cooled Nuclear Power Plants.
- 2.2.2 Design Criteria/Input Record. GAI W.O. #04-4283-070
- 2.2.3 Gilbert Associates, Inc. (GAI) Drawings:

Excavation and Grading Plan. E-774-151. 2231 2232 Plant Lavout E-012-006 F-014-004. E-430-006 2233 Structural. F-430-007 E-430-008 E-430-011 F-430-012 F-430-013 E-430-014 F-430-015 E-311-873 2.2.3.4 Building Services--Piping F-311-874 2235 Flectrical \$5-308-417 B-256-031 E-266-011

2.3 Design Basis

- 2.3.1 The SWSF is designed to comply with the requirements of RG.1.143, July, 1978. The facility is designed to provide a controlled, but ready access for material handling operations and to ensure that the operator exposures are as low as reasonably achievable (ALARA).
- 2.3.2 The facility is designed to maintain the dose rates in accordance with 10 CFR Part 20 and to meet the requirements of 40 CFR Part 190 at the site boundary and beyond.
- 2.3.3 The shielding thickness was calculated to limit the contact dose rates at the outer surfaces of the module walls and the top of the cell covers to within 0.5 mr/hr and 2.5 mr/hr, respectively. The analysis was based on the types of waste defined in Attachment 1. No credit was taken for the structure being partially underground.
- 2.3.4 Quality Assurance requirements for the design, operation and construction of the SWSF are consistent with those specified in Regulatory Guide 1.143.

2.4 Summary System Description

2.4.1 The concrete structure and individual cell covers provide the necessary shielding from the radioactive waste housed in the SWSF Module Storage Cells

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The cell covers with gaskets protect the waste containers from the elements and the ingress of precipitation. Slots and weep holes in the upper module structure are provided to direct rainwater to external drainage ditches. A drainage piping system prevents any spillage/leakage of fluids from accumulating in the cells (i.e., floor drain hub in each cell), the system manifold discharges into a common sume.

- 2.4.2 The sump compartment, a radwaste selsmic concrete structure, houses the pump, valves, piping, instrumentation, etc., necessary to perform the functions and control the disposal of any effluent which may collect in the sump. The compartment is divided into two levels, with the upper operator level shielded by a thick concrete floor from the sump. Access to the upper compartment is via a manhole in the concrete slab roof. Access to the sump is via a removable ladder at the sump, lug opening and a permanently installed ladder into the sump.
- 2.4.3 The flow diagram (Figure 3) shows the pumping system for the sump effluent. All operations are local/manual. The local alarms and sump level indication are housed in a weatherproof instrumentation panel mounted outside the Module "A" structure adjacent to the sump compartment.

The sump pump, Solenoid Valve #WS-5 and three-way valve #WS-1 (extension spindle) are located in the lower sumo while the electrical distribution and control panels, valves, etc. are mounted in the operators compartment.

Sump level is measured by a variable capacitance sensor (SNS-LE-O1) which transmits the signal to local and remote (Unit-2 Control Room) alarms.

Sump influent flow alarms are provided. The sensing elements (conductivity) Nos SNS-CE-07 and SNS-CE-08 are mounted in the Module "A" and "B" drain system manifolds.

2.4.3.1 Sump Pumping Operations (See Figure 3)

The SWSF sump is controlled and disposal of the effluent will be in accordance with Unit-2 Chemistry Procedure 4212-CHM-3011.99 and Operating Procedure 4212-OPS-3232.14. The sump compartment is posted as a radiological controlled area and surveillance is required prior to entry to ensure operator exposure will be as low as reasonably achievable (ALARA).

On receipt of the alarm signal (approximately 50% sump level) in Unit-2 Control Room, the above procedures are put into effect and the sequence of operations is as follows:

- 2.4.3.1.1 Recirculation Mode (Sump effluent mixing to obtain representative sample for analysis).
 - A. All valves to be checked closed.
 - Operate three-way Valve WS-1 to route pump discharge returned to sump.
 - C. Start pump.

2.4.3.1.2 Sampling Mode (during Recirculation Mode)

NOTE: Radiologically monitor the collection of the sample with the appropriate instrument.

- A. Solenoid Valve WS-5 open.
- B. Valve WS-7 open.
- C. Collect sample.
- D. WS-5 and WS-7 closed.
- E. Stop pump.

2.4.3.1.3 Discharge Mode (Only after sample analysis is completed).

- A. All valves to be checked closed.
- Connect hose to the tank truck (or portable vehicle) connection for effluent disposal.
- C. Operate three-way Valve WS-1 to route pump discharge to truck discharge manifold.
- D. Valve WS-4 (WS-3) open.
- E. Start pump.
- F. Stop pump, disconnect tank truck (or portable vehicle) connection, and allow contents of hose to drain back into sump.
- G. Valve WS-4 (or WS-3) closed.
- H. Operate three-way Valve WS-1 for recirculation mode and secure.
- 2.4.4 Major Equipment
 - A. Sump Pump (1.): Gould Model 3171 1 x 1-1/2 -6

Capacity	50 gpm
TDH	100 ft.
Fluid	Radioactive waste water/resin
	slurry pH approx. 7.
Materials	Cast iron/bronze fittings
Impeller	Open type
Discharge Conn.	Flanged above mtg. plate
Service	Intermittent 5 yr. life

B. Solenoid Valve (1):

1/2" nom. bore. 120 volt AC

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C. Instrumentation: See Table 1.

- 2.4.5 Facility General Arrangement: See typical layout Figure 2. The general arrangement, layout and details of the SWSF systems are shown in the drawings referenced in Section 2.2.3.
- 2.4.6 Instrumentation and Controls
- 2.4.6.1 The SWSF has three (3) instrument loops as follows:
 - A. A level instrument string provides the alarm and level indication both local and remote. In addition, this loop provides a sump pump permissive at greater than 10 percent level.
 - B. The other two (2) instrument strings are conductivity flow loops providing local alarms. One loop senses input from the drain discharge manifolds from Module "A" and the other from Module "B".
- 2.4.6.2 Instrument Setpoint Index. See Table 2.
- 2.4.7 System Interfaces

There are five interfaces associated with this facility:

- Processing Systems: Access road to and from waste and fluid processing facilities for transportation of materials and equipment.
- Material Handling System: Facility will accept radioactive waste containers from the processing systems and are compatible with the transportation and lifting equipment, i.e., transfer shield and site cranage (Manitowoc 4000 N mobile crane or equivalent).

 Cooling Water Pump (CWP) House: 480 V, 3 & 200 A feeder cable from Bus 2-61 shall provide power for the following:

A. 460 volts to the sump pump.

B. 480 volt welding receptacle.

- C. 25 KVA, 240-120V power center to energize lighting, convenience receptacles, instrumentation, and control devices.
- NOTE: No permanent heat tracing required for sump compartments.
- 4. Control Room: Sump level alarm.
- Chemistry Laboratories: Sump effluent samples for chemistry and radiological analysis prior to disposition. <u>NOTE</u>: The sump pump discharge is not directly connected to any plant systems, a local hose station is provided.
- 2.4.8 Operations-Radwaste Handling

The major operations performed at the SWSF is handling the radioactive waste containers while loading/unloading the individual cells in accordance with the types of containers like those specified in Section 2.1.3.

Unit 2 procedures, like those listed in Table 3 but not limited to those specifically written for these operations, are strictly adhered to, using the Manitowoc Mobile Crane or equivalent and appropriately shielded equipment. Each cell has an individual concrete cover 8'3" square x 3'0" deep. (Dwg. #B-430-015) weighing approximately Fourteen (14) tons. Normally only one (1) cover will be removed at any time from the cells containing radioactive waste containers within a module system. If more than one (1) cover is to be removed at any one time, additional safety precautions will have to be exercised.

2.4.9 Maintenance

Most operations including Maintenance requires a RWP.

Inner surfaces of the cells and the sump are epoxy coated to ease decontamination of the facility.

- 2.4.10 Acceptance Testing.
- 2.4.10.1 Mechanical. Dwgs. #E-311-873 and E-311-874
 - A. Module "A" and "B" Drain Piping Systems.
 - Leak Test in accordance with ANSI 831.1.1977. Criteria: Static Head. (Fill System, water level top of drain hubs) Holding Period. 10 minutes minimum. Acceptance. No visual leakage.

(ii) Flow verification, allow leak test water to drain to sump.

Criteria: No visible fluid in system.

B. Sump Pump "A" and associated piping.

Initial Service Leak Test in accordance with ANSI B31.1, 1977.

Criteria: Pump discharge pressure

Acceptance. No visual leakage, all welded joints

leaktight.

- A. Continuity and Megger tests were performed for all circuits.
- Instrument and Control were tested and calibrated in accordance with MIX 507.

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- C. Sump Pump "A", tested in accordance with Electrical Preoperational Test Procedure WG-E01.
- D. Solenoid Valve #HS-V05, tested in accordance with Electrical Preoperational Test Procedure WG-E02.

- 11 -Table 1

Solid Waste Staging Facility Instrumentation

Instrument Locations Functions Designator Model or Type SWS-LE-01 Drexelbrook 700-2-57 Mod A Sump Sump Level Sensor Mod A Opr. Floor Sump Level Transmitter SWS-LT-11 Drexelbrook 408-6230 370-1104-401 Mod A Opr. Floor Sump Level Indication SWS-LI-01 SWS-LI-01A International Instr. 1151 Mod A Top Panel Sump Level Indication SWS-LSL-01 SETCON 401-100x Mod A Top Panel Sump Level Switch Low (Pump Permissive) Mod A Top Panel Sump Level Switch High SWS-LSH-01 SETCON 401-100x SWS-LAH-01 PANALARM Mod A Top Panel Sump Level Alarm High SWS-LAH-01A PANALARM CR Panel 17E-24 Sump Level Alarm High SWS-CE-07 Level Lance 14-115V Sump Influent Flow Conductivity Element Influent Flow Alarm SWS-CAH-07 PANALARM Mod A Top Panel SWS-CE-08 Level Lance 14-115V Sump Influent Flow Conductivity Element SWS-CAH-08 PANALARM Mod A Top Panel Influent Flow Alarm

Table 2

Instrument Setpoint Index

Instrument Tag No.	Instrument Desc	ription	Component Type	Setpoint Descrip.	Action	
SMS-LSL-01 (GAI: LB-S-4)	Solid Waste Sta Switch Low (Sump	ging Facility Sump Level p Pump Permissive)	Current Switch	10% level (5.6 ma) Increasing	Contact	Closure
SWS-LSH-01 (GAI: LB-S-3)	Solid Waste Star Switch High	ging Facility Sump Level	Current Switch	42% level (11.2 ma) Increasing	Contact	Closure
SMS-CE-07 (GAI: CE-S-7)	Solid Waste Star Flow Module 'A'	ging Facility Sump Influent Conductivity Element	Conductivity Element	Maximum Sensitivity	Contact	Closure
SWS-CE-08 (CAI: CE-S-8)	Solid Waste Star Flow Module 'B&G	ging Facility Sump Influent C' Conductivity Element	Conductivity Element	Maximum Sensitivity	Contact	Closure

Table 3

OPERATING PROCEDURES

Proc. No.	Description
4215-OPS-3232.14	SWSF sump pumping operation.
2104.4.103	EPICOR II 4' \star 4' liner transfer and shipping cask loading.
4212-0PS-4450.08	On-site transfer of Radioactive liners to SWSF/Ship.
4214-OPS-4450.09	On-site transfer of Radioactive liners from SWSF.
4214-0PS-4440.01	Transfer of EPICOR 4' x 4' liners to SWSF/Ship.

- 14 -Attachment 1

Shielding Analysis - Types of Waste

NOTE: SDS vessels used in the SDS processing system may not be stored in this facility unless resin has been removed.

Types of waste considered are given below. A 3 month decay period was used in the analysis.

- Natural Circulation Evaporator with Solidification Waste Form: 55 gallon drums (solidified) Design Basis for Cell: C-D waste at VR* = 4.5, n** = .6
 18 drums per storage cell
 C-D waste analysis is given below
- or 2) Forced Circulation Evaporator/Crystallizer with Solidification Waste Form: 55 gallon drum (solidified) Design Basis for Cell: C-D waste at VR = 22, n = .6 18 drums per storage cell
- or 3) Epicor II Charcoal Filter

Waste Form: Activated Charcoal in 4 ft. diameter x 4 ft. high liner Design Basis for Cell: 2500 R/hr on contact Two liners per cell

or 4) Epicor II Demineralizer Resins

Haste Form: Dewatered Resins in 4 ft. diameter x 4 ft. high liner Design Basis for Cell: B-C waste at VR = 543 B-C waste analysis is given below

- *VR volume reduction
- ** n Packaging efficiency: ratio of volume of waste to total container volume.
- A) Quantities; C-D Waste

83,000 gallon - Reactor Coolant Bleed Tank - A 83,000 gallon - Reactor Coolant Bleed Tank - B 250,000 gallon - Reactor Building Sump

B) Isotopic Analysis (µCi/ml) - Design Basis; C-D Waste

Isotope µCi/ml

Mo 99	- 180	Cs	136	-	120	Ce 144 - 100
1 131	- 8200	Ba	140	-	290	H 3-1.2
I 132	- 20	La	140	-	160	
Cs 134	- 82	Sr	89	-	1400	
Cs 137	- 330	Sr	90	-	120	

()

Isotopic Analysis (µCi/ml) - Design Basis; B-C Waste

B-C Waste

Isotope	µCi/ml	Isotope	µCi/ml

Ba 133	2 E - 1	Ba 140	7.5 E - 1
Co 60	1.4 E - 4	La 140	2.4 E + 0
Cs 134	1.6 E - 1		
Mo 99	2.7 E - 1		
I 131	2.3 E + 1		
Bi 207	2.2 E - 2		
Co 58	1.2 E - 3		
Ce 127	7 5 1		







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